

## The initiation of stress corrosion crack in Alloy 718 under pressurized water reactor primary water conditions

**Haipeng Zhu**<sup>1,2</sup>, Zhiming Zhang<sup>3</sup>, Jianqiu Wang<sup>1,2,3</sup>, En-hou Han<sup>1,3</sup>

<sup>1</sup>CAS Key Laboratory of Nuclear Materials and Safety Assessment, Institute of Metal Research, Chinese Academy of Sciences, Shenyang, 110016, P. R. China

<sup>2</sup>School of Materials Science and Engineering, University of Science and Technology of China, Shenyang, 110016, P. R. China

<sup>3</sup>Institute of Corrosion Science and Technology, Guangzhou, 510530, P. R. China

hpzhu20s@imr.ac.cn

**Abstract** Stress corrosion crack initiation behavior of alloy 718 was investigated through slow strain rate tensile (SSRT) test. Alloy 718 was subjected to two commercial heat treatments. Specimens solution-treated at 1080 °C exhibit precipitation of the  $\gamma'$  and  $\gamma''$  phase, ensuring the material possesses outstanding mechanical properties. This heat treatment method is characterized by a low sensitivity to stress corrosion. Conversely, specimens subjected to solid solution treatment at 970 °C demonstrate an increased propensity for stress corrosion cracking due to the additional precipitation of the  $\delta$  phase. Grain boundaries, MX phases, and delta phases are susceptible to preferential oxidation, making them the prime sites for crack initiation.

**Keywords** Alloy 718; Stress corrosion cracking; Pressurized water reactor

### Reference

[1] Z. Ning, H. Zhang, S. Zhang, X. Hao, E.-H. Han, W. Kuang, Effects of precipitation on the stress corrosion crack initiation of alloy 718 in simulated pressurized water reactor primary environment, *Acta Materialia* 276 (2024).

[2] M. Wang, M. Song, G.S. Was, J.L. Nelson, The roles of thermal mechanical treatment and  $\delta$  phase in the stress corrosion cracking of alloy 718 in primary water, *Corrosion Science* 160 (2019).

[3] K.J. Leonard, M.N. Gussev, J.N. Stevens, J.T. Busby, Analysis of stress corrosion cracking in alloy 718 following commercial reactor exposure, *J Nucl Mater* 466 (2015) 443-459.